

NTEC Module: Water Reactor Performance and Safety
**Lecture 8: Loss-of-coolant accident
 (LOCA) phenomena**

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Reactor Operational States

- **NORMAL OPERATION:** Operation at full power.
- **OPERATIONAL TRANSIENTS:** Startup and shutdown. On-line refuelling (AGR).
- **UPSET CONDITIONS:** Unexpected faults, e.g. turbine trips. Loss of offsite power.
- **EMERGENCY CONDITIONS:** Break in small pipe, relief valve stuck open etc.
- **LIMITING FAULT CONDITIONS (DESIGN BASIS ACCIDENTS):** Design to cope with by engineered safety systems, e.g. large pipe break.

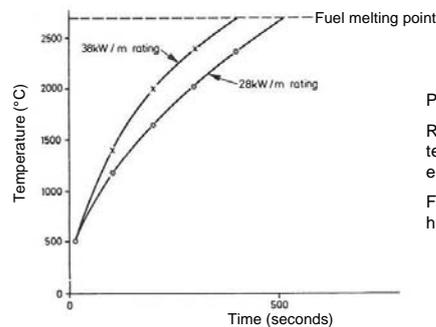
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Frequency of occurrence of reactor operational states

Operating states for which the system is designed to cope:	Frequency of occurrence
Normal operation	Continuous (apart from shutdowns for maintenance)
Operational transients	~ 10 per reactor year
Upsets	~ 1 per reactor year
Emergencies	1 in 100 reactor years
Limiting fault conditions (including design basis accident, DBA)	1 in 10,000 reactor years
Unprotected or beyond design basis accidents	1 in 1 million reactor years

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Adiabatic heating of fuel elements after reactor trip



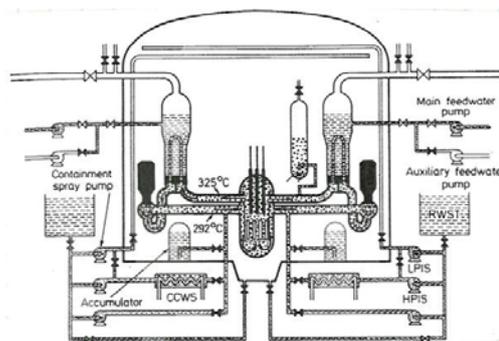
PWR fuel elements.
 Rapid initial rise and temperature equalises.
 Fission product heating continues.

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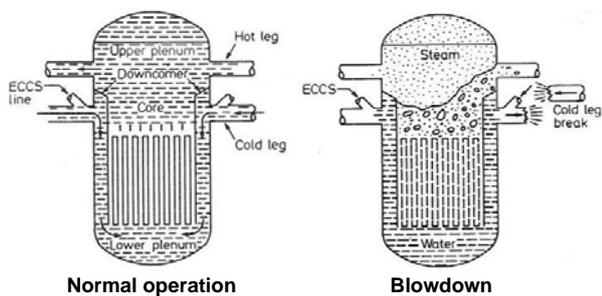
Events in fuel element behaviour as temperature increases

Temperature (°C)	Phenomenon
350	Approximate cladding temperature during power operation.
800 – 1500	Cladding is perforated or swells as a result of rod internal gas pressure in the post-accident environment; some fission gases are released; solid reactions between stainless steels and Zircaloy begin; clad swelling may block some flow channels.
1450 – 1500	Zircaloy steam reaction may produce energy in excess of decay heat; gas absorption embrittles Zircaloy, hydrogen formed. Steel alloy melts.
1550 – 1650	Zircaloy-steam reaction may be autocatalytic unless Zircaloy is quenched by immersion.
1900	Zircaloy melts, fission product release from UO ₂ becomes increasingly significant above 2150 K.
2700	UO ₂ and ZrO ₂ melt.

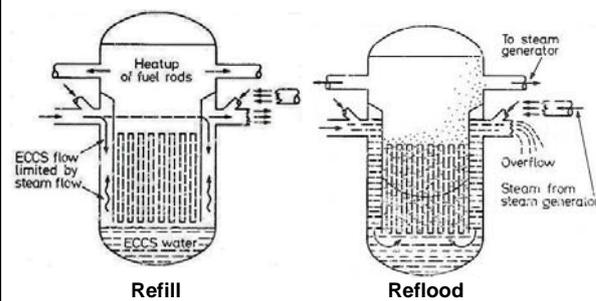
Emergency Core Cooling System (ECCS) for PWR reactor



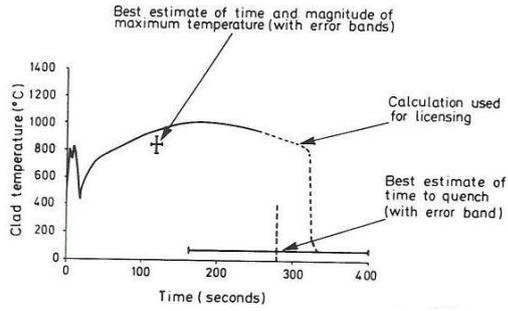
Events in a LARGE BREAK LOCA in a PWR (I)



Events in a LARGE BREAK LOCA in a PWR (II)

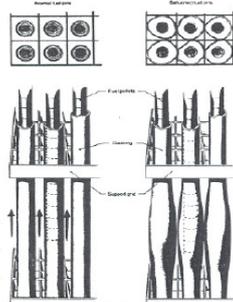


Variation of peak clad temperature with time in Large Break PWR LOCA



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A complicating feature: Clad ballooning



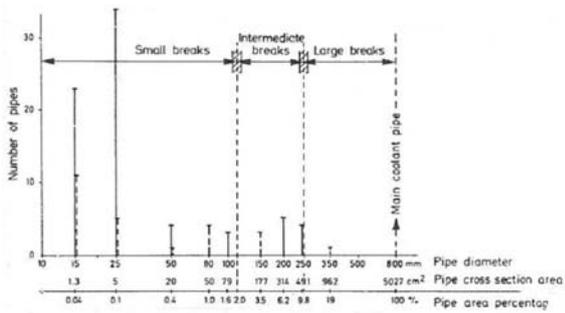
Fuel elements in PWR are pressurised.

At temperatures above ca. 800 C clad swells and may block flow.

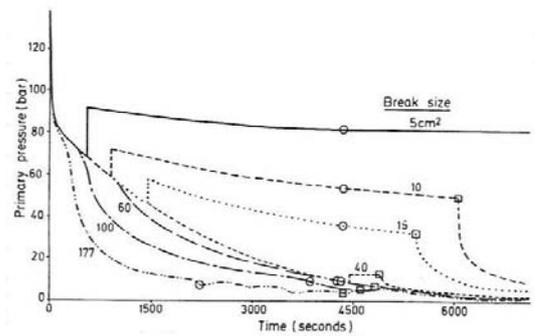
Further discussion in Lecture 10.

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Small break LOCAs: Pipe Sizes

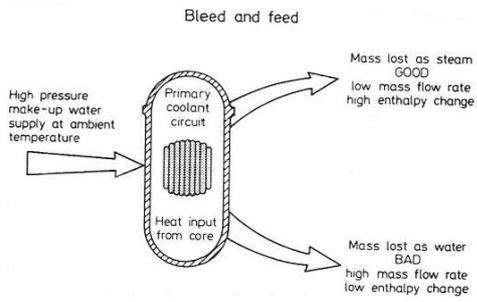


Small break LOCAs: Variation of pressure with time



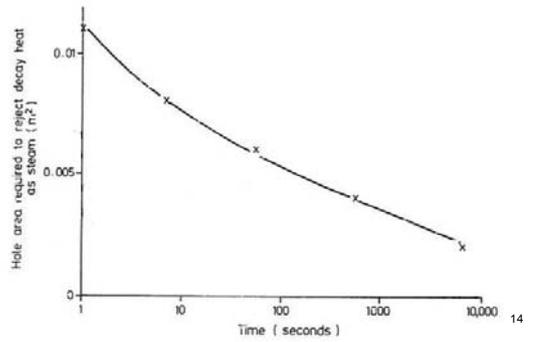
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Small break LOCAs: Energy outflows



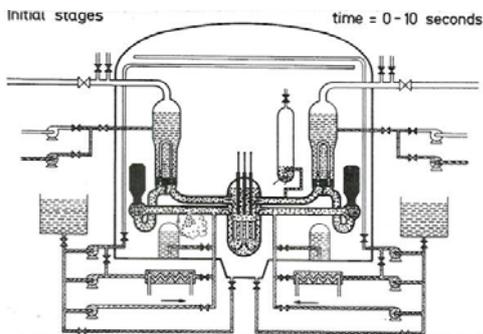
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Small break LOCAs: Hole size to remove energy as steam



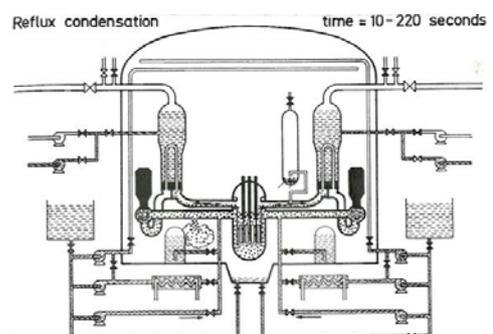
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Small break LOCAs: Initial phase



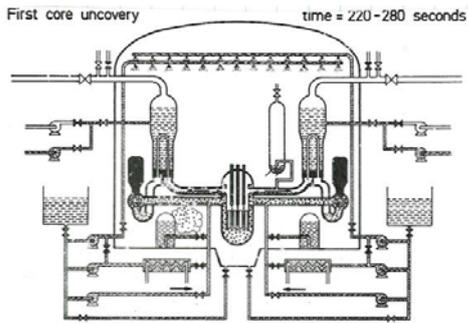
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Small break LOCAs: Reflux condensation

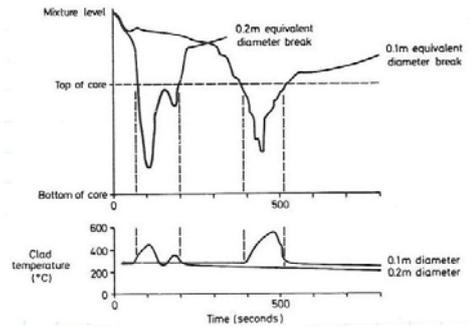


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Small break LOCAs: Core uncover



Small break LOCAs: Mixture level and clad temperature



Conclusion

- Design over full range of operational states is vital.
- What seems to be the extreme case is sometimes not the most important (e.g. small break rather than large break LOCA's are worst case!)
- It is important to analyse and learn as much as possible from actual accidents.